

A large, stylized atomic symbol in the background on the left side of the slide, composed of concentric green circles and dots.

NATrIUM

Advanced Reactor Demonstration Project Overview

a TerraPower & GE-Hitachi technology

Advanced Reactor Demonstration Program (ARDP)

demonstration goals

- Demonstrate the ability to design, license, construct, startup and operate our Natrium™ Advanced Reactor
- Build the supply chain for sodium fast reactors in the United States
- Lower emissions by initiating the deployment of a fleet of Natrium reactors – Demo will show that we can build these plants economically and that they will be attractive for future owner/operators
- Development of new jobs and a stronger economy

Elements of the Natrium™ ARDP Offering

- Strong private financial backing
- Design, NRC Part 50 licensing, Procurement, Construction and Startup of Natrium Demonstration reactor 840 MWt through fuel loading within 7 years
- Support Development of HALEU metal fuel supply chain
 - Providing funding for first year of development in two cascades of 120 AC100M centrifuges that would add 12 MTU/year of capacity at 19.75 wt. % U²³⁵.
 - Design, license and construct a fuel fabrication facility for HALEU metallic fuel
- Continue development of TP advanced fuel and material development and qualification program

Team Members



HITACHI



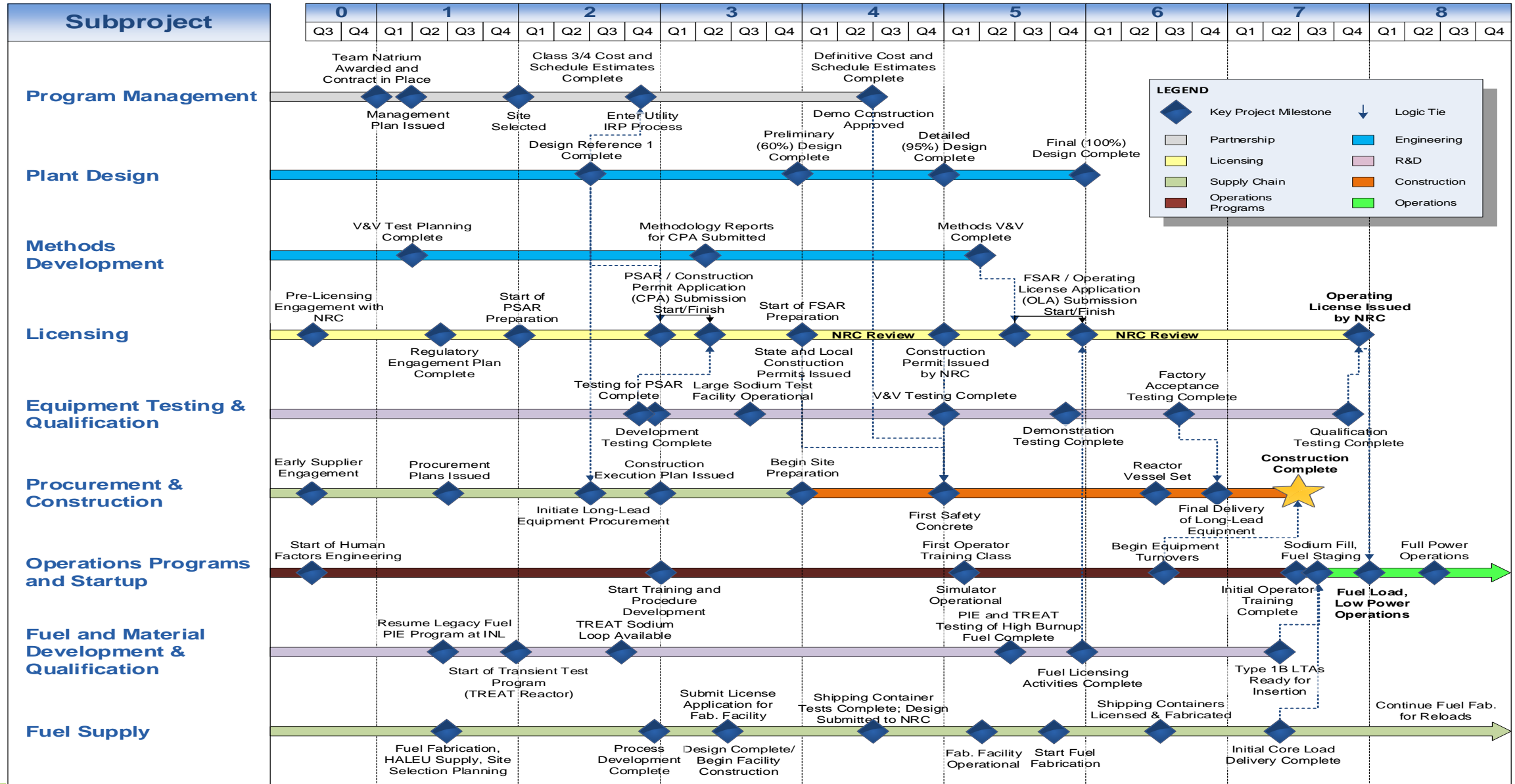
Why Natrium™ technology?

It is the best option for deploying advanced nuclear power given increasing demand for carbon-free energy and a growing mix of renewable sources because it:

- Simplifies construction and architecture compared to previous reactor types
- Offers a cost-competitive, flexible technology that supports load following, energy storage and industrial process heat applications
- Brings step-change improvements in capital and operational costs
- Provides a utility-scale decarbonization solution that can make a meaningful impact on efforts to mitigate climate change

Team Natrium

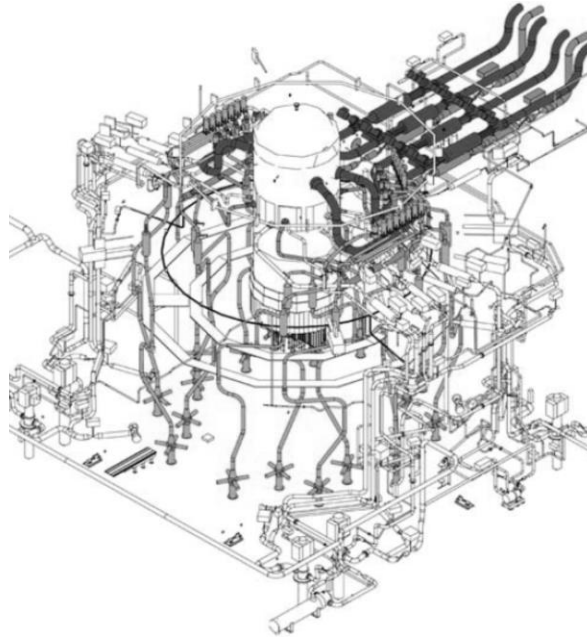
ARDP Natrium Demonstration Project Level 1 Schedule



Schedule and Cost Estimate Methodology

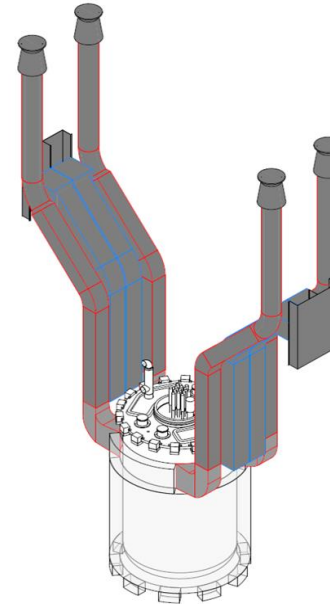
- Assumes a defined spending plan with consistent source of funds
- Jointly developed by TP, GEH and Bechtel based on past experiences
- Licensing schedule has been discussed several times with NRC "aggressive but achievable"
- Design to minimize construction time
- AACE Class 3 / 4 cost estimate and schedule by end of budget period 2
- AACE Definitive cost estimate and schedule for construction will be provided in 4th budget period

Simple Nuclear Systems



LWR Emergency Core Cooling

- 2600+ ASME Sect. III Pipe Welds
- High Pressure Injection (1000+ PSI)
- Large Water Inventory Requirements
- Active Valve and Pump Operation
- Multiple Trains and Sub-systems

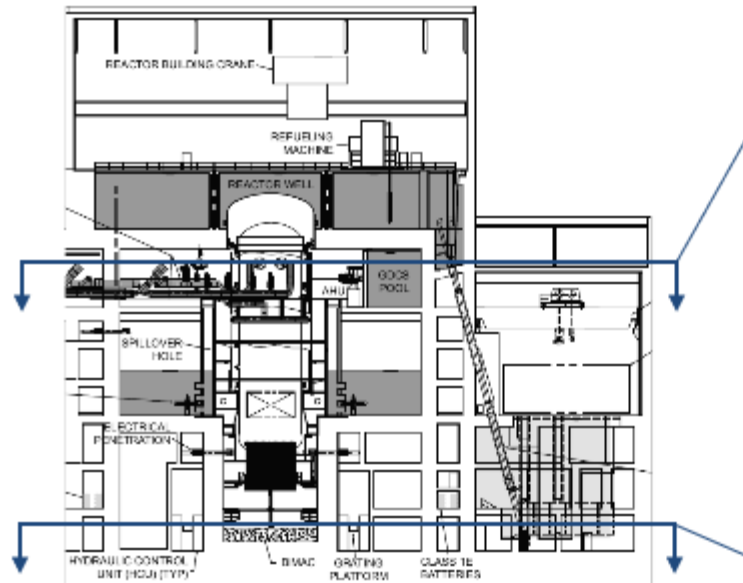


Sodium Radiant Vessel Cooling

- Zero ASME Sect. III Pipe Welds
- Atmospheric Pressure (<1 PSI)
- Unlimited Air-Cooled Heat Sink Supply
- Fully Passive (Always in Operation)
- Singular Rugged System

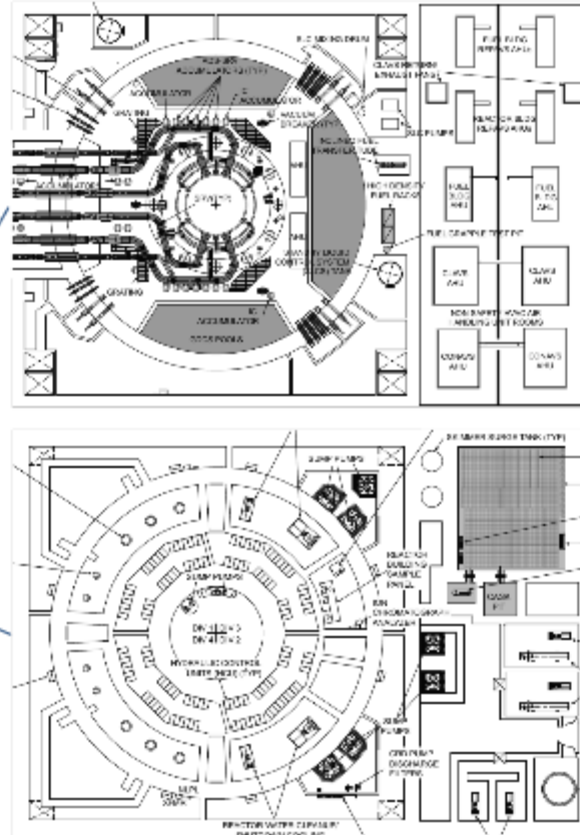
Simple Nuclear Buildings

Light Water Reactor (LWR) ~105 m³/MWe total nuclear concrete



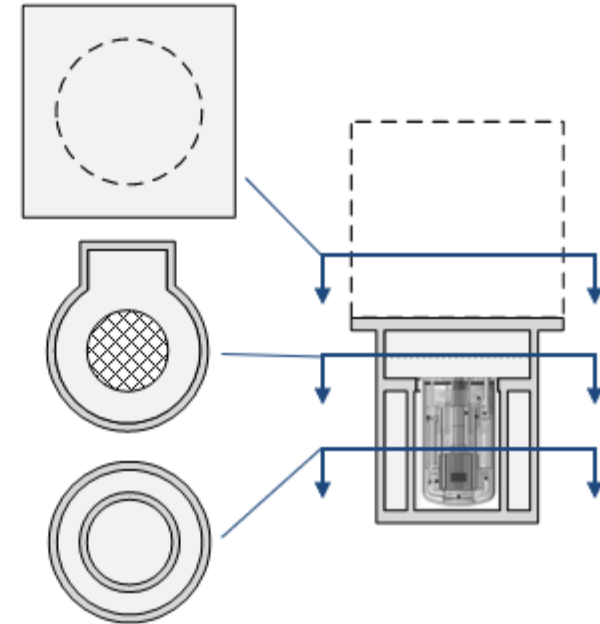
LWR Reactor Building

- Sprawling equipment
- Compartmentalized
- Heavily congested



Shown to Scale

Sodium ~20 m³/MWe total nuclear concrete



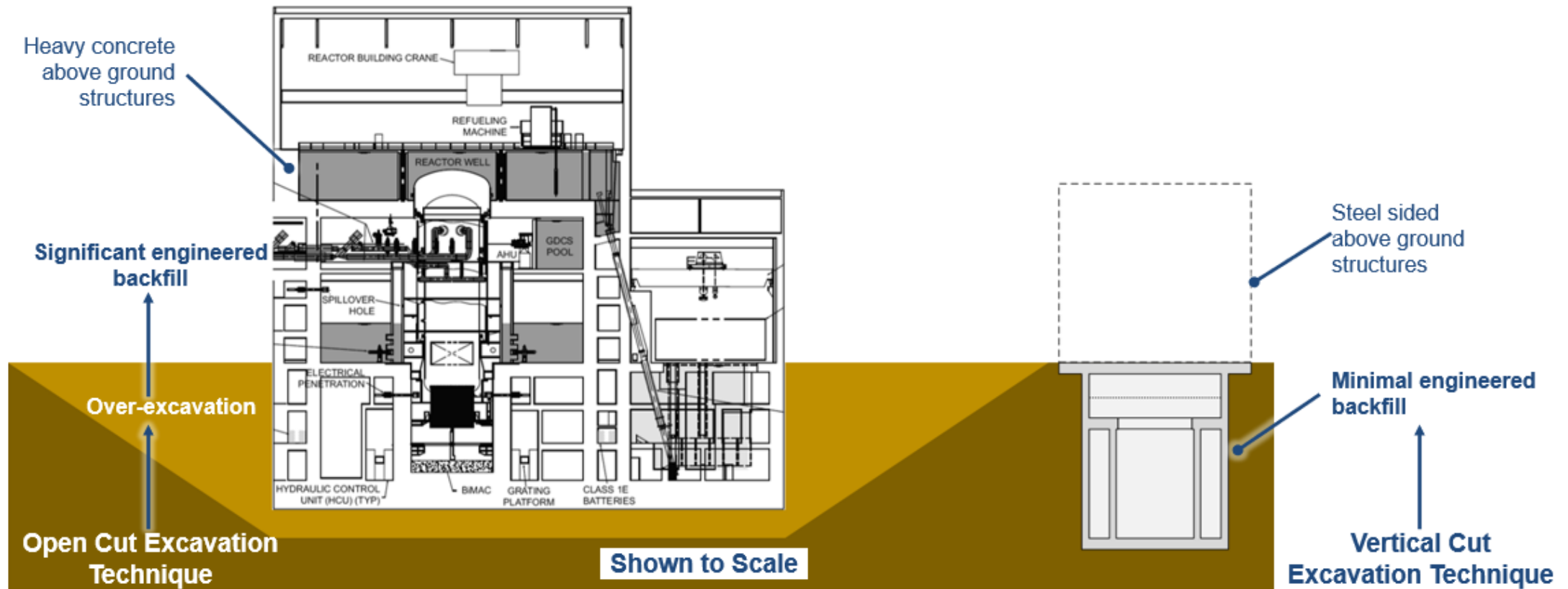
Sodium Reactor Building

- Compact pool reactor equipment
- Few internal walls
- Open floor plan

Simple Nuclear Construction

Light Water Reactor (LWR)

Sodium



Customers?

- Marketing to US Utilities
- About 10 US Utilities have joined our Utility Advisory Board
- In discussions with a potential owner/operators for the demonstration plant
- Key success criteria –
 - Sodium can be located anywhere that meets the NRC criteria, but has added value in markets that have a high penetration of renewables, that both need a stable baseload and need power on demand when solar and wind are not available.
 - Strong political support, especially at the state and local levels
 - Strong workforce
 - EPRI Siting Guide

Licensing Strategy

- Team Natrium selected the 10 CFR 50 licensing process due to the time requirements on the program. 10 CFR 50 allows you to start construction earlier.
- The Preliminary Safety Analysis Report will be submitted with a Part 50 Construction Permit Application (CPA). The PSAR is currently organized consistent with the draft PSAR/FSAR outline and guidance provided by an NRC letter dated April 15, 2020, titled "Updated Draft Outline for Licensing Modernization Project Advanced Reactor License Applications"
- The Final Safety Analysis Report will be submitted with the Operating License Application

Natrium Reactor Planned NRC Interactions Environmental (2021-2022)

- Site Selection Process
- Environmental Coordination State and Federal
- Environmental Need for Power and Alternate Analysis
- Severe Accident Mitigation Analysis
- Environmental – Fuel Cycle Impacts

Planned NRC Interactions Safety/Programs (2021-2027)

- **Quality Assurance – NRC currently reviewing**
- **Safeguards Information (SGI) Plan**
- **Probabilistic Risk Assessment (PRA)**
- **Sodium Demo Plant**
- **Applicability of Regulations**
- **Engineering Computer Codes for Sodium**
- **M++ (Mongoose sub channel code) Thermal Hydraulic Topical**
- **Risk Informed Performance Based Preliminary Design Criteria**
- **Plant-Level Risk-Informed Performance-Based SSC's Classification**
- **Energy Island Decoupling**
- **Consensus Codes and Standards**
- **Source Term Methodology**
- **Design Basis Accident Methodology**
- **PSAR Chapter 1 – Plant and site overview**
- **Licensing Basis Events with and without releases**
- **Emergency Preparedness**
- **Metal Fuel Operating Experience**
- **Radiological Release Consequences Methodology**
- **Reactor Stability Methodology**
- **Partial Flow Blockage Methodology**
- **Regulatory Exemptions**
- **Fuel Methodology**

Key Licensing Milestones (2023 – 2028)

- Construction Permit Application and PSAR Submittal – August 2023
- License to Construct – March 2025
- Operating License Application and FSAR Submittal – March 2026
- License of Operate – March 2028

Redefining what nuclear can be...

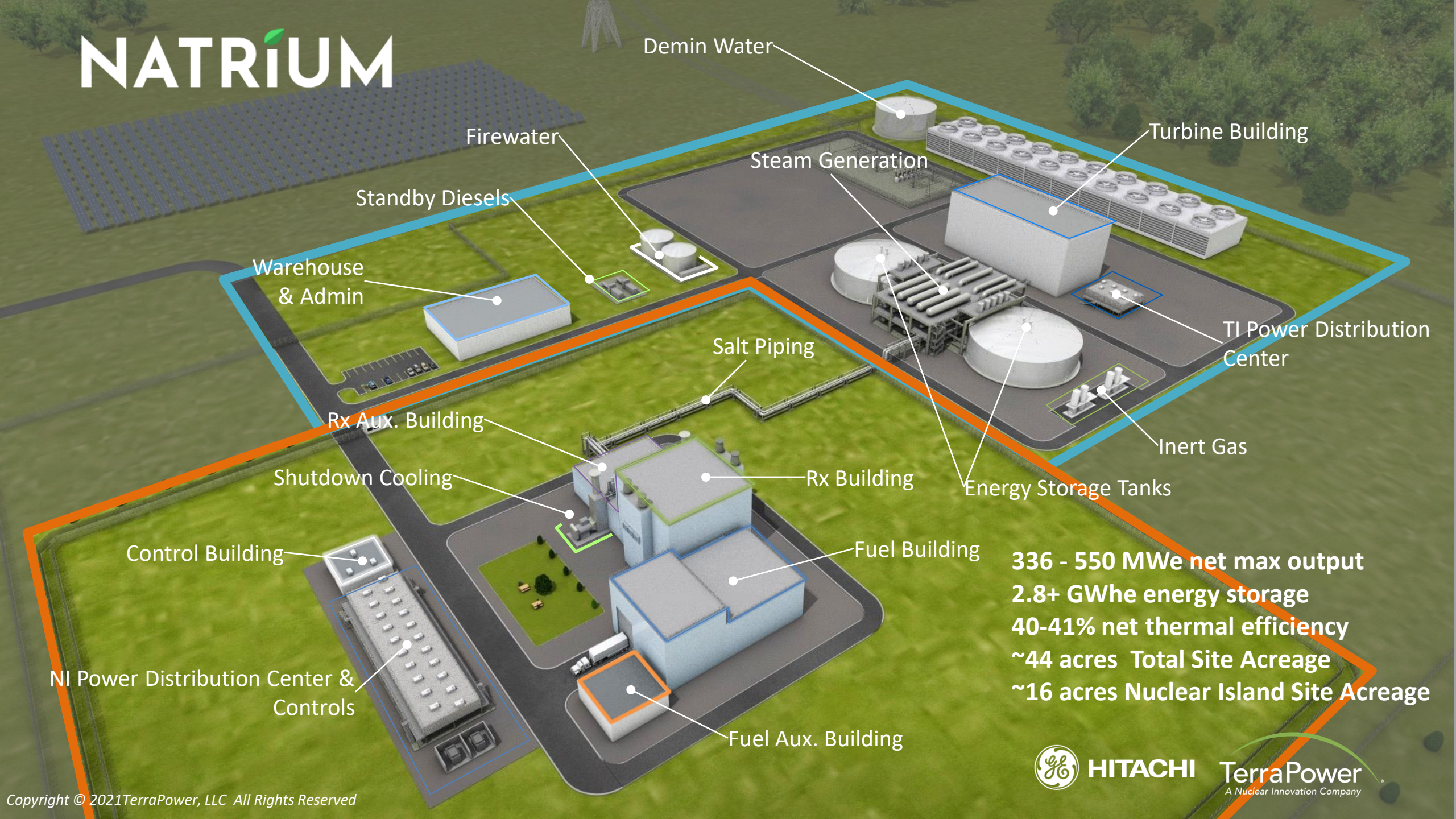
Nuclear redefined

- Eliminates nuclear “sprawl”
 - ✓ Design to cost
 - ✓ Simplicity
 - ✓ Rapid construction
 - ✓ Design specific staffing
- ~41% net thermal efficiency

Integrating with renewables

- Zero emission dispatchable resource
- Price follower... w/ reactor at 100% power 24/7
- 345 MWe nominal
- Flex to 500 MWe for 5.5 hours through energy storage

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336 - 550 MWe net max output
2.8+ GWhe energy storage
40-41% net thermal efficiency
~44 acres Total Site Acreage
~16 acres Nuclear Island Site Acreage



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TerraPower
A Nuclear Innovation Company

Heat Transport Architecture

It's all about the tanks.

How Energy Storage Works

Basic Operation

Charging: Low Price $\dot{W}_T < \eta_T \dot{Q}_{Rx}$

- Hot salt tank level increases
- Cold salt tank level decreases

Discharging: High Price $\dot{W}_T > \eta_T \dot{Q}_{Rx}$

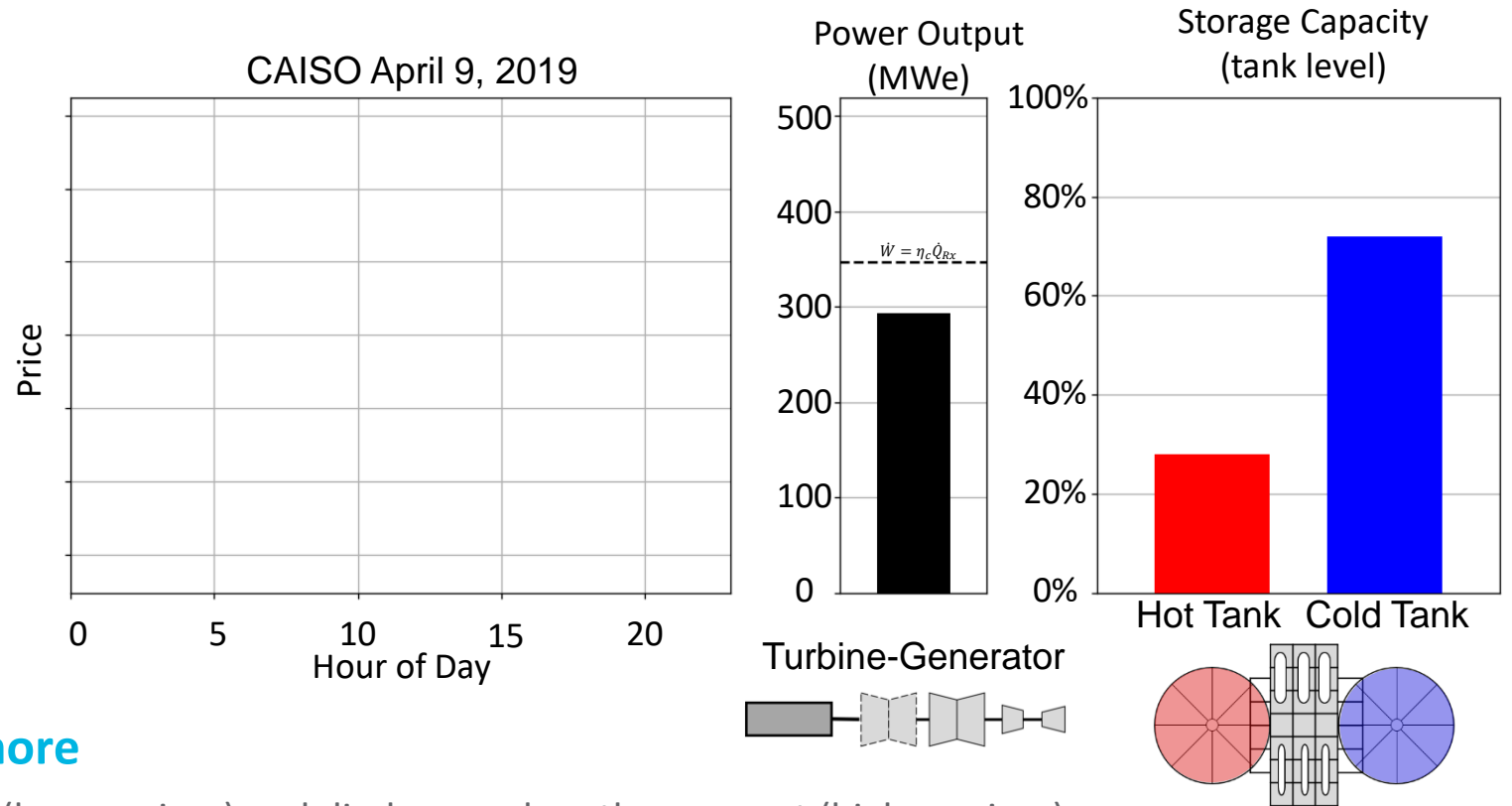
- Hot tank salt level decreases
- Cold tank salt level increases

Even: $\dot{W}_T = \eta_T \dot{Q}_{Rx}$

- Steady hot tank salt level
- Steady cold tank salt level

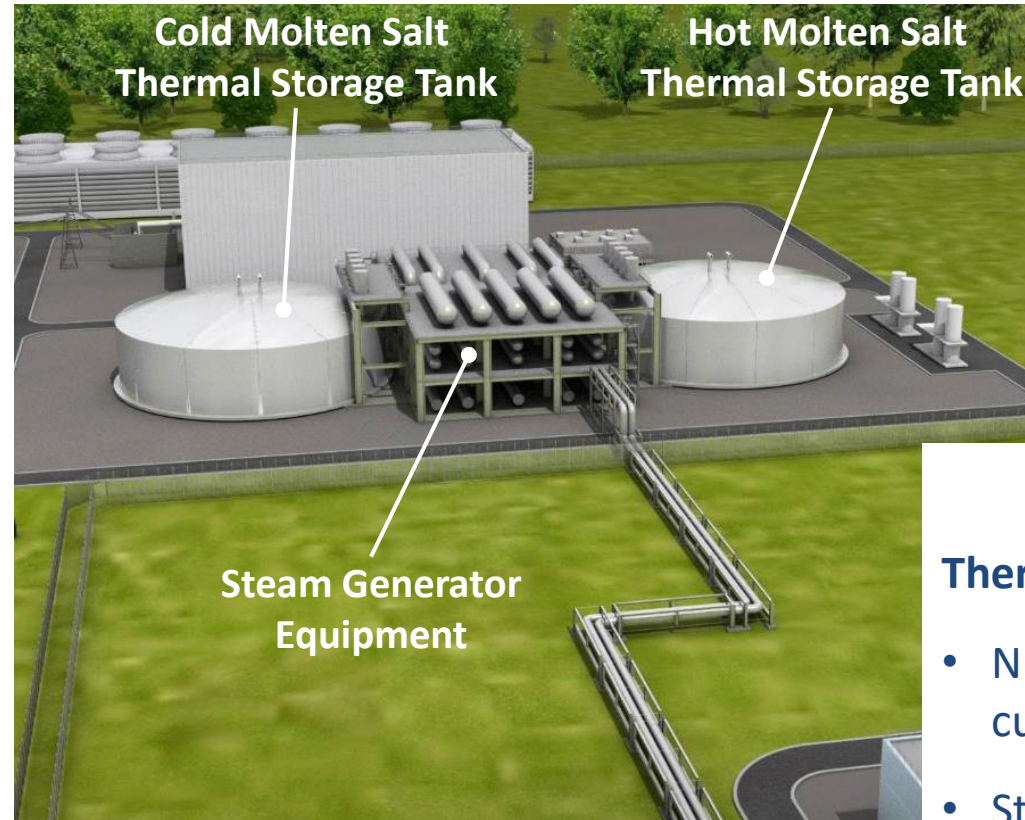
Low price - sell less, high price – sell more

- Store when renewables producing power (lower prices) and discharge when they are not (higher prices)
- Sodium is different from LWRs because the outlet temperature is high enough to support storage.
- Reactor output is steady ... minimize cycling of reactor
- Price following above and below 100% reactor power



$$\text{Full Power Turbine Hours} = \frac{(\rho V_{\text{working}})_{\text{hot}} (C_{\text{hot}} T_{\text{hot}} - C_{\text{cold}} T_{\text{cold}})}{\frac{\dot{Q}_T}{\eta_T} - \dot{Q}_{Rx}}$$

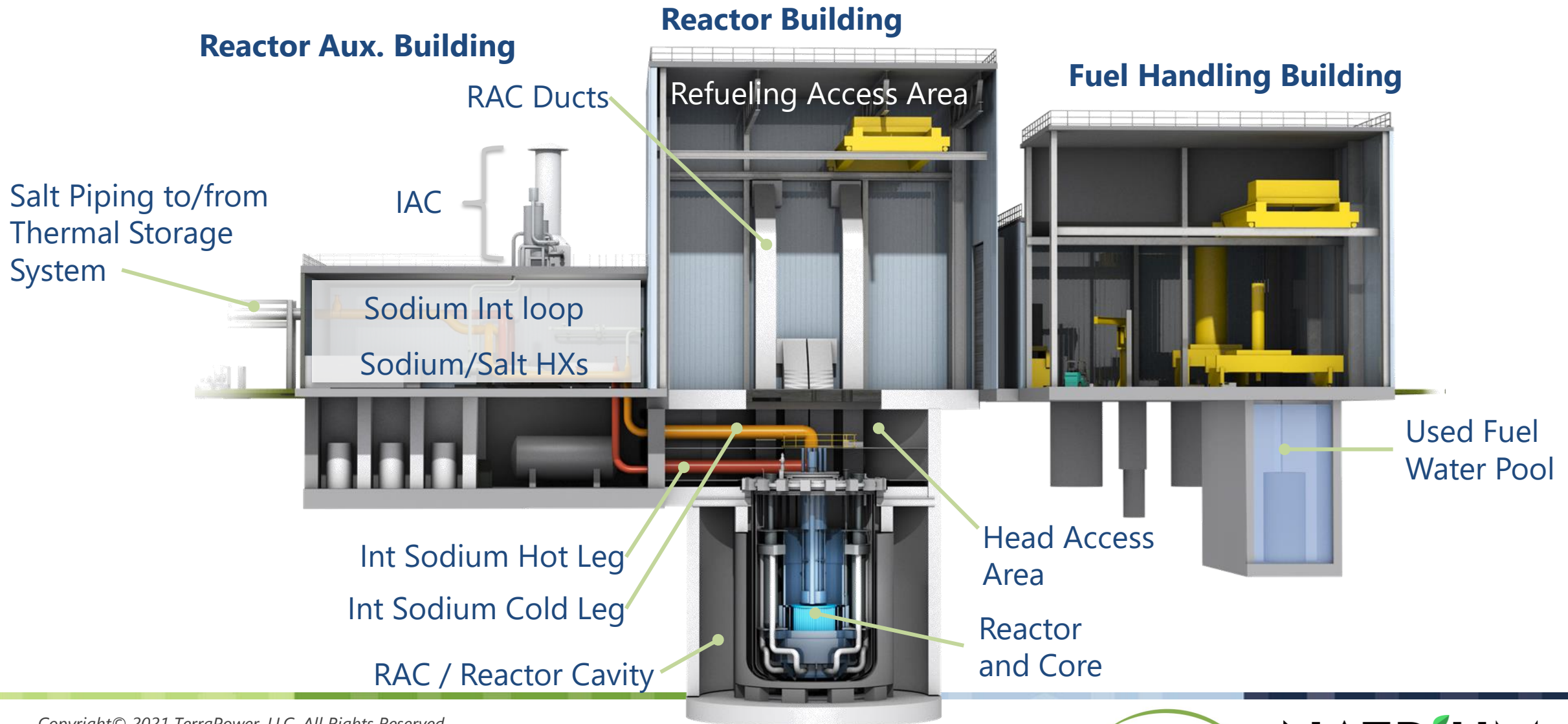
Thermal Storage



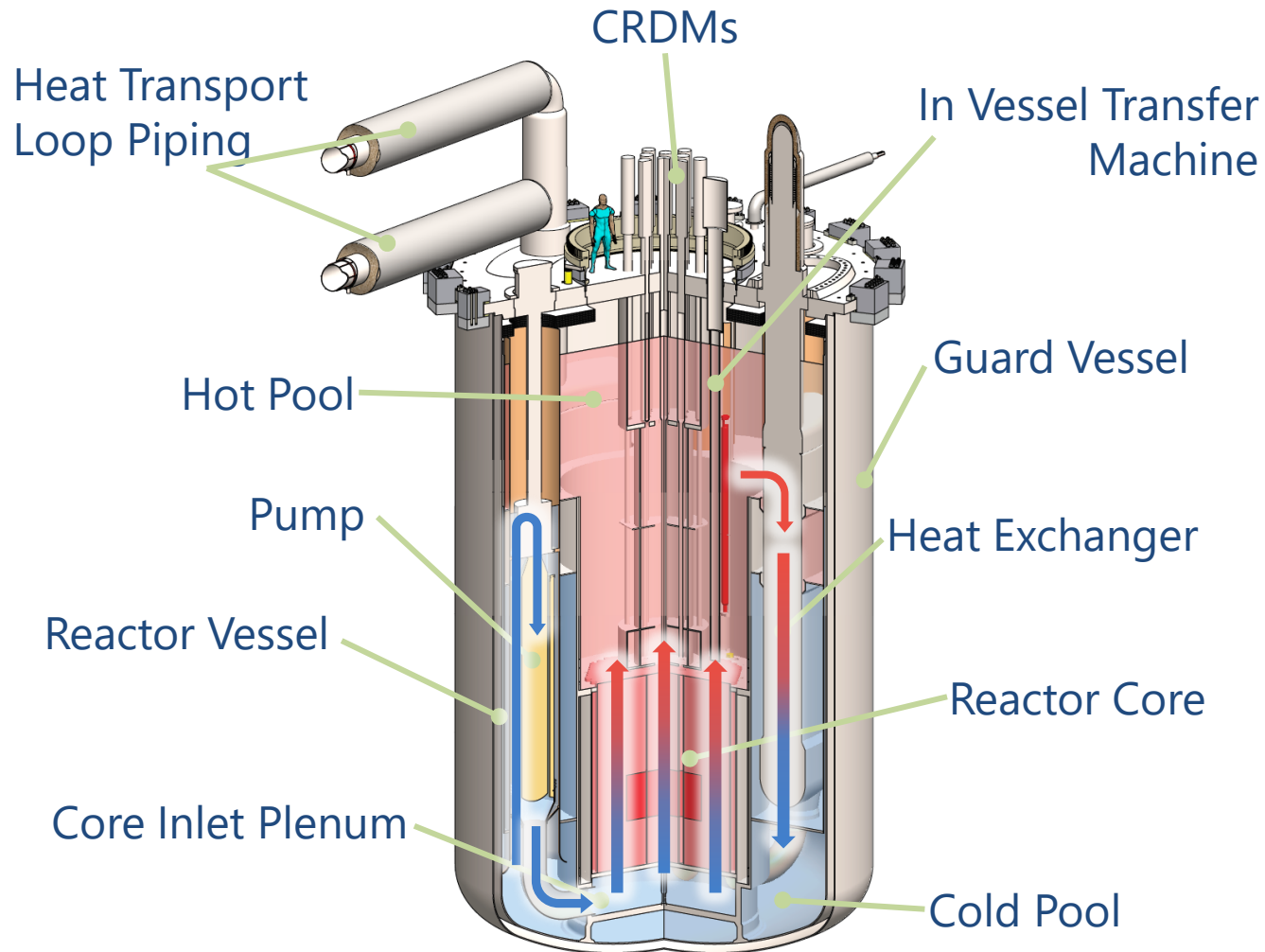
Thermal Storage

- Number of tanks based on customer's energy need
- Steam generator trains based on size of turbines
- Turbine size based on customer's power need

Key Technical Features of Sodium Reactor Buildings



Key Technical Features of Reactor Equipment Design



- Advanced Fuel System
 - long lived, once through, lower cost fuel
- Efficient Transport Architecture
 - Molten Salt Loop transports heat to Energy Island
- Simple Reactor Structures
 - Safety related / nuclear seismic sub-structures
 - Open floor plan: few internal walls
- Compact Reactor Equipment
 - Pantograph In-Vessel Fuel Handling Machine
 - Space efficient internals design for easy fabrication
- Natural Circulation (Air) based safety cooling
 - RAC simplifies safety analysis & defense
- Local Loop Cooling for Refueling (non safety)
 - Na to air heat exchangers on Intermediate Loop

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Simple Nuclear System

- Exceptional heat transfer
- Passive air cooling
- Low pressure
- Optimized layout

Flexible

- Dispatchable power
- Energy storage... price follow
- Integrate with renewables
- Process heat

Competitive
Clean Energy

Adjacent innovations

- Concentrated solar power industry
- Tunneling industry... vertical cut
- Combined cycle gas turbine industry